# Absorption of Radiation

Lecture 28 www.physics.uoguelph.ca/~pgarrett/teaching.html

#### Review of L-26

- $\beta$ 's loose energy in matter via 3 main processes
  - Ionization
  - Bremsstrahlung
  - Annihilation
- α's interact with matter mainly through ionization
- A quantity called *range* is defined as  $range = penetration depth \times density$
- For kinetic energy of  $\beta$ 's (or  $e^-$ ) > 0.6 MeV, empirical relation  $Range(\frac{kg}{m^2}) = 5.42E 1.33$

where E is in MeV

• Distance of penetration D for  $\alpha$  particle of energy E (MeV) in air is  $D(cm) \approx 0.325E^{\frac{3}{2}}$ 

## Review of L-26

- Main processes for  $\gamma$  rays interacting with matter
  - Coherent scattering
  - Compton scattering
  - Photoelectric effect
  - Pair production
  - Photonuclear reactions

# Linear energy transfer (*LET*)

- Particles can lose energy continuously along their path when interacting with matter, and so continuously slow down
- When working out the impact to biological systems (or matter), the amount of energy lost by the particle per unit distance, called the *linear* energy transfer, or LET, is important

$$LET = \frac{dE}{dx}$$

- Particles with a large *LET* can do a large amount of damage, but usually over a very short range
- Particles with a small *LET* do lesser damage, but can affect tissues further from the source
- The energy of a particle is a nonlinear function of the distance travelled, so the *LET* can vary enormously over a particles path
- Very often the *LET* is largest towards the *end* of a particles path, so radiation treatments for cancer "tune" energies of, e.g.  $e^-$  beams so that the  $e^-$  stop in tumours

#### Particles and *LET*

- Heavy charged particles and  $\alpha$ 's have a very high *LET*
- $e^{\pm}$  have a *LET* that varies from small for high energies, to large for low energies
  - This is what makes *e* beams useful for cancer therapies
  - Do minimal amount of damage to tissue at entry point, maximum damage to tumour
- Protons have a larger *LET* than  $e^-$ , but smaller than  $\alpha$ 's
  - Also used in cancer therapies
- Other particles used in beam therapies for cancer treatment
  - Neutrons
  - Muons like "heavy" electrons
  - Pions
  - Accelerated ions
- Each has its advantages/disadvantages
- By far most popular is X-ray beam therapy since every hospital has X-ray facility, but not always particle accelerator

## Absorption of radiation

- Radiation classified in two main ways
  - Ionizing (charged particles, UV, X-rays, γ-rays, neutrons,...
  - Non-ionizing (microwaves, radiowaves, ...)
- Most damaging form of radiation are those causing ionization
- Ionizing radiation quantified by amount of ionization the radiation causes in dry air at NTP
- Physical processes that deposit energy in matter lead to ionization if an electron positive-ion pair can be created
  - In dry air, this requires ~35 eV of energy (at NTP)
  - ex. An electron of energy 5 MeV is stopped in air at NTP, how many  $e^-$ -ion pairs are created?
    - $n = 5 \times 10^6 / 35 \approx 143000$  ion pairs

### Exposure

- The strength of the radiation field is quantified by the exposure – the amount of radiation that will produce charge of either sign in dry air at NTP
- Exposure units are Coulombs/kg
  - Unit is *Röntgen* (R)
  - $-1 R = 2.58 \times 10^{-4} C/kg$
- Ex. A radiation field causes the total production of 10<sup>9</sup> ion-pairs per kg of air
  - Exposure =  $10^9 \times 1.602 \times 10^{-19} = 1.6 \times 10^{-10} \,\text{C/kg} = 0.62 \,\mu\text{R}$
- Very often, exposure is given in form of a rate, i.e. R/hr
  - If the same field now causes  $10^9$  ion pairs/s,  $0.62 \,\mu\text{R/s} = 2.23 \,\text{mR/hr}$

#### Radiation dose

- *Dose* = amount of energy absorbed per unit mass
  - SI unit Gray (Gy) = 1 J/kg
  - Old unit rad = 0.01 J/kg
- For charged particles, dose calculations straightforward (add up energy of all particles emitted divide by total mass of matter)
- For photons, more difficult (since photons are not slowed down and stopped by matter, but attenuated and scattered)
- Photon attenuation is (works for  $\gamma$  and X and UV,...)

$$I = I_0 e^{-\mu x}$$

μ is attenuation coefficient

Need energy absorption

# Absorption coefficient

- Photon scattering photon can leave volume without depositing all its energy
- Absorption coefficient takes into account photoelectric, pair, Compton absorption
- Scattering coefficient takes into account scattered photons
- Attenuation coefficient = absorption + scattering
- Very often, quantities expressed as mass attenuation, absorption,... coefficients

$$\mu_m = \frac{\mu}{\rho}$$
 m<sup>2</sup>/kg

with  $\rho$  density

• For  $\gamma$  rays with 100 keV<E $_{\gamma}$ < 5 MeV,  $\mu_m$  ~ 3.0×10<sup>-3</sup> m²/kg

#### Absorbed dose

• For photons, dose (D) is

$$D \approx 3.0 \times 10^{-3} NEt$$
 J/kg

E is the photon energy (J), N is photon rate (#/s), t the time

- Ex. A 1  $\mu$ Ci source is ingested that is both a  $\beta$  and  $\gamma$  emitter, and spreads uniformly over the body of a 60 kg person. The average  $\beta$  energy is 1 MeV, and for each decay, a 1.5 MeV  $\gamma$  is emitted. What is the dose rate?
  - The β energy is totally absorbed =  $1 \text{ MeV} \times 1 \times 10^{-6} \times 3.7 \times 10^{10}$ =  $3.7 \times 10^4 \text{ MeV/s} \implies 617 \text{ MeV/kg/s}$  (divide by body mass)
  - The  $\gamma$  energy absorbed

$$D_{\gamma}$$
  $s^{-1} = 3.0 \times 10^{-3} \times 3.7 \times 10^{4} \times 1.5 = 166.5$  MeV/kg/s

- Dose rate =  $45 \mu rad/hr$ 

# Biological half life

- Body processes ingested materials at specific rates, depending on the chemistry and metabolism
- Elimination of materials by body can be described by an exponential with a *biological* half life  $t_{1/2}^b$
- Radionuclide ingested by body
  - Amount left after time t is

$$N_b(t) = N_0 e^{-\lambda t} e^{-\lambda_b t} = N_0 e^{-\lambda_{eff} t}$$

where

$$\lambda_{eff} = \lambda + \lambda_b = \frac{t_{1/2} + t_{1/2}^b}{t_{1/2} t_{1/2}^b}$$

#### Total dose

- Take our previous example, work out the total dose assuming a half life of 5 yr, and biological half life of 6 days
- $\lambda_{\rm eff} \approx \ln 2 / 6 \, \mathrm{d}^{-1}$
- At time t, the # decays that occur in the body in interval dt is A dt (A is the activity =  $\lambda N$ )
- Total # decays in the body is

$$\int_0^T A dt = \int_0^T \lambda N_0 e^{-\lambda_{eff} t} dt = \frac{\lambda}{\lambda_{eff}} N_0 \left( 1 - e^{-\lambda_{eff} T} \right)$$

• With our example of a 1  $\mu$ Ci source, #decays=37000×6×24×3600/ln(2) = 2.77 ×10<sup>10</sup>, and the total dose is 2.77×10<sup>10</sup>×(1/60 ( $\beta$ 's)+0.003( $\gamma$ 's))×1.6×10<sup>-13</sup> =8.72×10<sup>-5</sup>=87  $\mu$ Gy=8.7 mrad

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## Dose equivalent

- Different radiations have different consequences on biological tissues
- Take this into account by giving each type of radiation a weighting factor so that they can be compared to the equivalent dose of X-rays
  - Relative biological effectiveness (RBE) or radiation weighting factor  $(w_R)$  these are just numbers
- Taking into account the weighting factor, we have the dose equivalent *H* 
  - SI unit Sievert (Sv)
  - Old unit rem 100 rem = 1 Sv

$$H = \sum w_R \times D$$

So our previous example,  $\beta$  and  $\gamma$  have  $w_R = 1$ , so the dose equivalent is 87  $\mu$ Sv or 8.7 mrem